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Research Article

**Use of the Natural Circulation Flow Map
Circulation Systems Evaluation**

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Abstract

The aim of this paper is to collect and resume the work done to bu engineering tool related to the natural circulation. After a brief des phase and two phase, the derivation of a suitable tool to judge the Finally, an extensive comparison among the NC performance of vai is done to show a practical application of the NC flow map.

1. Introduction

The natural circulation (NC) is an important mechanism in sev behaviour is of interest to nuclear reactor design, operation, and true for new concepts that largely exploit the gravity forces for the the presence of a heat source and of a heat sink constituted in a ni generators, respectively. In a gravity environment, with the cc generators, driving forces occur such that to generate suitable flow NC core power removal capability was only exploited for accider safety features of the plants, recently within the passive reac importance having a large impact on the design phase.

The NC behaviour has been (and it is still now) object of several experimental or characterization test in a test rig. In this view, a quite large data base of measurements from various facilities. The NC scenarios occurring in the inventory were considered (reference is made to both single-phase and two-phase) and analyzing data from the following PWR simulators: Semiscale, etc.

In order to evaluate the natural circulation performance (NCP) index, the data comes from the analysis of the trend of the core inlet mass flow rate and the residual masses have been normalised taking in account the corresponding power level (typically ranging between 1 and 5% of the design power). Four main flow patterns were characterized depending on the primary loop.

2. The Natural Circulation Flow Regimes

2.1. Single-Phase NC (SPNC)

SPNC regime implies no void occurrence in the upper plenum of the reactor core. The core is subcooled up to nearly saturated. Core flow rate is derived from the balance of driving forces and resistant forces. Driving forces are the result of fluid density differences occurring in the downcomer and the core and ascending side of U-tubes. Resistant forces are the friction along the entire loop. Resulting fluid velocities are sufficient for the natural or forced convection heat transfer regimes: no film boiling conditions occur. The secondary side of SG is also a natural circulation system working in the SPNC regime.

SPNC may occur at any primary system pressure, consistently with the design pressures range between 8 and 16 MPa with secondary pressure expected from the NPP design that SPNC, provided the availability of decay heat from the core. Experimental database, including NPP tests, is available.

2.2. Stable Two-Phase NC (TPNC)

TPNC regime occurs as a consequence of coolant loss from the primary system. The resistant forces increase when decreasing mass inventory of primary system. The former effect, that is, increase of driving forces, is predominant for larger decreases of mass inventories. The net driving force is positive (when primary mass flow rate decreases) and natural convection, subcooled, and saturated heat transfer regimes occur in the primary system. The average core void fraction is typically less than 30%, reached without occurrence of thermal crisis in the considered pressure range.



Figure 1: Characterization of natural circulation flow regimes based on system code calculations.

2.3. Siphon Condensation NC (SCNC)

The decreasing of NC driving forces, the small temperature difference between the core and the secondary side of SG, leads to the SCNC regime.

the occurrence of the countercurrent flow limiting phenomenon (CCFL) at the origin of wide system oscillations of core inlet flow rate. This is based on a natural circulation experiment performed in Lobi 1. Steam condensation has been found also in other facilities.

At mass inventories of the primary system around 70% of the nominal power transfer across U-tubes causes the release of almost all core thermal energy. The liquid level builds up and is prevented to drain down by the steam-liquid interface. A CCFL condition occurs. Therefore, liquid level rises in the U-tubes. After a duration of the order of 10 seconds, the flow rate at the core inlet drops to zero. When the liquid level reaches the upper bend of U-tubes, the siphon effect occurs on the cold side of U-tubes and the reestablishment of core inlet flow rate. A complex interaction of the several thousands of U-tubes may stay at a different stage of the oscillation at the same time. Suitable core cooling still can be achieved in these conditions.

2.4. Reflux Condensation (RCNC)

At “low” mass inventories of primary coolant and/or at low core inlet flow rate including hot legs and steam generator entrance is low. When the mass inventories are not enough to cause CCFL. In these conditions, the liquid level on the cold side of the U-tubes may flow back to the hot leg and to the core inlet. This occurs simultaneously in the hot legs. The mass flow rate at the core inlet is zero. A natural circulation path may establish between core and downcomer inside the hot legs and downward liquid flows occur at the core outlet. Core thermal power is low. A nucleate boiling heat transfer regime.

2.5. Dryout Occurrence

The terms “dryout occurrence” appear in the right part of Figure 1. When the mass inventories are lower than 40% of the nominal value. Dryout is caused by the consequence of CCFL, film boiling heat transfer regime is experienced when the liquid level rises. The temperature increases in various zones of the core, and the overall system operation may become unstable. The system operation in these conditions is not safe. It can be noted that the temperature excursion is limited by the maximum thermal power levels: the linear rod power plays a role in these conditions. At 15 MPa (nominal operation for PWR), “post-dryout” surface temperature is not tolerable for the mechanical resistance of the rod-clad material.

All the regimes cited above are summarized in Figure 1 which represents the vertical axis the percentage of the nominal core power (P) is reported on the horizontal axis. The mass inventory of the primary circuit (RM) is reported on the horizontal axis.

3. The Natural Circulation Flow Map

The database gathered from ten experiments performed in the simulator has been used ([6, 7]) to establish a natural circulation flow map.

Table 1: Relevant hardware characteristic data for the natural circulation flow map creation.

In all the considered ITF, NC experiments with similar modalities h in the previous chapter are experienced. The linear power of the power, and the primary system pressure constitute the main differ experiments. In relation to the primary side pressure, PKL experi roughly half of the value adopted in the other facilities. The rang (e.g., pipe diameter, system volume, number of the steam genera discussed here, and the identified differences are assumed to prod typical PWR when decay heat removal is concerned.

Measured values of core inlet flow rate (G , kg/s), core power (P , M and net volume of the primary system ($V = \text{const.}$, m^3) have b map (NCFM). The diagram G/P versus RM/V has been preferred i nondimensional quantities.

The experimental database from ITF (six ITFs, ten experiments) ar and 3, respectively. The envelope in Figure 3 is assumed to constit

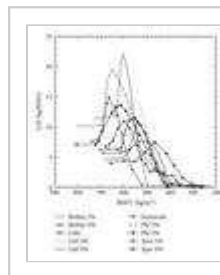


Figure 2: Natural circulation system behavior PWR simulators.

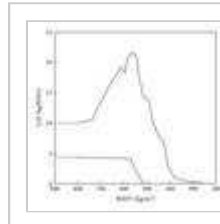


Figure 3: Natural circulation flow map achieved simulator.

A first demonstration of the use of the NCFM has been done in [8 ITFs, not used for setting up the database presented in Figure 2, I and ITF can be drawn from Tables 2 and 3, respectively. The c horizontal SG design. The ITFs are Pactel and RD14M (see Tab CANDU NPP, respectively. Their geometric layout is different from loops equipped with HTSG are connected to the vessel, though c configuration characterizes the CANDU design, that is, equipped wi

ITF	Power (MW)	Flow (kg/s)	Pressure (MPa)
1	100	1000	15
2	150	1500	15
3	200	2000	15
4	250	2500	15
5	300	3000	15
6	350	3500	15

Table 2: Relevant characteristics of NPP consid

Parameter	PACTEL	RD14M
Power (MW)	100	150
Flow (kg/s)	1000	1500
Pressure (MPa)	15	15

Table 3: PACTEL and RD14M.

Comparing the calculated data with the NCFM, authors concluded t

- (i) PWR equipped by OTSG have poor natural circulation per
- (ii) NPP designed around passive system concept showed a q
- (iii) Russian design reactors equipped by horizontal SG are al
- (iv) RD-14m database was not fully qualified (e.g., the docun

large deviation from the map.

The NCFM was used to characterize the behaviour of the CNA-I PH mass inventory scenario [6]. The simulations have been performed in detail. The coarser one (SET I) was used as a basic set just to refer to (4). This permitted to verify that the trends known for most ITFs with CNA-I behaviour, considering appropriate trips of some of its safety

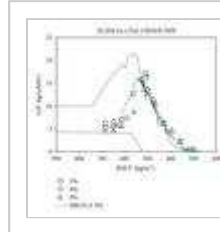


Figure 4: Natural circulation map for Atucha SBLOCA.

4. Recent Use of the NC Flow Map

4.1. Application to the PSB-VVER Facility

As already showed by the previous examples (other can be regarded as an advantageous tool to depict the NC capability in removing the core heat), several experiments have been done that cover practically all the NPP types.

In Figure 5 (taken from [10]) it can be seen the use of the NCFM for the PSB-VVER facility. The experiment has been carried out in the frame of an LBLOCA test (B-9401) carried out in the framework of an IAEA project. In this test, the pressure are maintained constant, while the mass is stepwise drawn out (in violet) are reported. The experimental data goes out from the map, indicating that the flow were not qualified in two-phase flow giving a wrong indication. This demonstrates the suitability of the NCFM and on the other hand the good NC performance of the PSB-VVER.

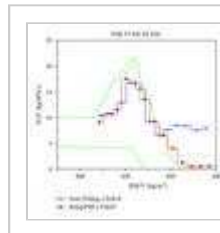


Figure 5: Natural circulation map for the PSB-VVER facility.

4.2. Revisiting the RD-14m Facility

In Section 3, the RD14m data were considered but achieving results with the NCFM calculation has been repeated fixing constant the core power and stepwise reduced, following a typical way to perform such kind of LBLOCA test (B-9401) carried out in the framework of an IAEA project. In Figures 6 and 7), two main conclusions can be drawn.

- (i) The CANDU facility seems not so suitable for NC status. The NCFM reasonably resumes the SG NC, it does not consider any of its definition.
- (ii) Figure 7 shows that in the CANDU installation other type of flow seems relevant from the core power removal point of view. The flow rate is greater than 500 kg/m^3 , where the NCFM predicts low flow (the core power) the dryout is not experienced.

Regarding the latter point (i.e., the channel to channel NC mode) over power versus time of two different horizontal channels. It can be seen that the mass flow rate (i.e., the water follows the normal flow path enter the channel), the other is cooled by water circulating in the opposite direction. The total core flow rate (sum of the core channels mass flow rate) reaches the nominal value.

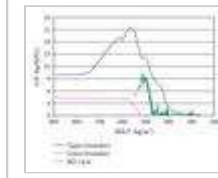


Figure 6: Natural circulation map for the RD-1

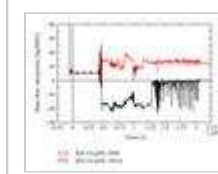


Figure 7: Channel-to-channel natural circulation

4.3. Consideration of BWR Data

The last use of the NCFM regards the consideration of boiling water reactor (BWR) and calculation results (RBMK).

Figure 8 reports the typical power-flow map of a BWR in which the power in NC mode is roughly 40% with a core flow rate of 25%.

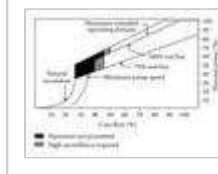


Figure 8: Typical BWR power-flow map.

In Figure 9 the BWR data are reported into the NCFM (green dots map: full power condition (green square) at full pump speed, 25% power). It can be seen that the points are practically aligned along a line, confirming the suitability of the BWR to work in NC mode. Value conditions in PWR simulators are experienced in BWR but with a performance in core removable power typical of PWR is not present at its nominal power.

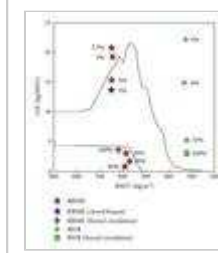


Figure 9: BWR and RBMK data and comparison

Again in Figure 9 it can be seen also the RBMK data. Those calculations use of a qualified nodalization developed and tested during an EPR project. It comes out when calculated data are put inside the NCFM and compared

- (i) The RBMK data stay almost aligned at various power levels.
- (ii) The full power point of both boiling water reactors (v (practically) same G/P ratio, but differs by a factor of two in type (vessel type versus channel type reactor), the difference in primary side (PS) mass of the two steam drums used in the RBMK (iii) The G/P ratio remains comparable between the two reactors.
- (iv) Deviation from the consideration of the item above is expected when the suction header is closed (blue star in Figure 9) at core power. The former case has an important negative impact on natural circulation. The latter case seems to represent the last possible NC operational point.
- (v) When power level around 3% is considered, the two reactors show a lower mass flow rate in the RBMK enhancing a better performance.

5. Conclusions

A system scaling study was the precursor of this work in which the NCFM tool aimed at evaluating the NC performance of an NPP. The NCFM was validated against PWR simulators.

Recently, the use of the NCFM has been extended to practically all experimental data available after an NC experiment carried out at 14m data after its nodalization qualification; BWR and RBMK comparisons.

Once more, the NCFM results in a very helpful tool to judge the NC performance. It can be noted that the NCFM is suitable to catch the SG NC type hence the channel-to-channel circulation experienced in the RD-14m. Considerations have been derived, other applications may regard support on scaling analysis and validation of computational tools.

As mentioned above, this engineering tool has been directly derived from a theoretical and generalized relationship between the system in question and the objective of a future research work.

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